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The CIELO Collaboration: Neutron Reactions on $^1\mathrm{H}, ^{16}\mathrm{O}, ^{56}\mathrm{Fe}, ^{235,238}\mathrm{U},$ and $^{239}\mathrm{Pu}$

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CIELO (Collaborative International Evaluated Library Organization) provides a new working paradigm to facilitate evaluated nuclear reaction data advances. It brings together experts from across the international nuclear reaction data community to identify and document discrepancies among existing evaluated data libraries, measured data, and model calculation interpretations, and aims to make progress in reconciling these discrepancies to create more accurate ENDF-formatted files. The focus will initially be on a small number of the highest-priority isotopes, namely ¹H, ¹⁶O, ⁵⁶Fe, ^{235,238}U, and ²³⁹Pu. This paper identifies discrepancies between various evaluations of the highest priority isotopes, and was commissioned by the OECD's Nuclear Energy Agency WPEC (Working Party on International Nuclear Data Evaluation Co-operation) during a meeting held in May 2012. The evaluated data for these materials in the existing nuclear data libraries — ENDF/B-VII.1, JEFF-3.1, JENDL-4.0, CENDL-3.1, ROSFOND, IRDFF 1.0 — are reviewed, discrepancies are identified, and some integral properties are given. The paper summarizes a program of nuclear science and computational work needed to create the new CIELO nuclear data evaluations.

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ттт	CEDITORIDAL MAREDIALC	7	Outstanding progress has been made around the	
	STRUCTURAL MATERIALS	7	in nuclear reaction and decay data evaluation.	
	A. ⁵⁶ Fe	7	quality of the main evaluated data libraries, s	
	1. Summary of the Evaluations	7	ENDF/B-VII.1 [1], JEFF-3.1 [2, 3], JENDL-4	[4]
	2. Inelastic Scattering	7	BROND/ROSFOND [5], and CENDL-3.1 [6], is high	
	3. (n,xn) and (n,xp) Reactions	8	for the most part the libraries perform well in ne	
	4. (n,α) Reaction	8	ics simulations for fission and fusion energy applie	
	4.00=22.22=0	_	(though covariance data that represent uncertaint	
IV.	ACTINIDES	9	less advanced). However, our current understanding	
	A. General Comments	9	sufficient in many essential areas, some user needs	
	B. Compensating Errors	9	inadequately addressed, and a new working parace	
	C. Fission Cross Sections	9	needed to expedite future evaluated nuclear reaction	
	D. Prompt Fission Neutron Spectra	10	advances. We see this as being facilitated by (a) p	
	E. Prompt Fission Gamma-ray Spectra	10	expertise from across the world through creation	
	F. Inelastic Scattering	10	laborative teams, and (b) using new computations	
	G. Radiative Capture	11	niques for optimization, sensitivity analyses, and	
	Н. ²³⁵ U	11	tainty quantification (UQ). Stronger international	
	1. Resolved Resonance Parameters	11	orations will provide a new framework for nuclea	
	2. Radiative Capture	11	evaluation, and will help establish the highest fideli	
	3. Inelastic Scattering	12	eral purpose nuclear database for all nuclear science	ce com-
	4. (n,2n) Reaction	12	munities around the world.	
	5. Average Number of Neutrons per Fission	10	It is recognized that for many important applic	
	$\overline{\nu}$	12	for example nuclear criticality calculations, the e	
	6. PFNS Integral Validation I. $^{238}\mathrm{U}$	13	evaluated data perform well in transport simulation	
		14	ing, in part, to compensating errors in the data	
	1. Radiative Capture	14	Different cross section libraries may predict almo	
	2. Elastic and Inelastic Scattering	14	same k_{eff} for benchmark experiments, but for ve	
	3. (n, 2n) Reaction4. Average Number of Neutrons per Fission	14	ferent reasons at a microscopic level [7, 8]. Su	
	9	15	rors must be minimized since simulation prediction	
	$\overline{ u}$ J. $^{239}\mathrm{Pu}$	15 15	from calibration points (corresponding to the bend	
	1. Resolved Resonance Parameters		experiments) can rapidly become erroneous if the	
	2. Radiative Capture	15 15	lying physical data used in a simulation are inc	
	3. Inelastic Scattering	16	Also, cross sections for transmutation reactions,	
	4. $(n, 2n)$ Reaction	16	ing fission, capture, and $(n, 2n)$, are inadequately	
	5. Average Number of Neutrons per Fission	10	for certain applications. And in many cases sca	
	$\overline{\nu}$	16	cross sections — elastic and inelastic, and secondar	
	6. PFNS Integral Validation	16	tron energy and angular distributions — are inaded	quatery
	o. 1110 integral validation	10	known for transport calculations. In this paper we suggest that a new paradigm is	noodod
V	VALIDATION BENCHMARKS	17	to more rapidly advance our understanding for th	
٧.	VIED THOU DENOTIFIED	11	uation of nuclear reaction cross sections. Closer in	
VΤ	ANALYSIS OF INTEGRAL QUANTITIES	17	tional cooperation is needed, where the world's ϵ	
v 1.	A. Thermal Cross Sections	18	for various capabilities are brought together to so	
	B. Westcott Factors	18	problems and to provide peer review on propos	
	C. Resonance Integrals	18	lutions. We suggest the name for this collaborate	
	C. 100001101100 1110081010	10	140101110. THE BUSSELV CHE HUMBE TO THIS COMMENTAL	01 V OI

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