



## Measurement of $^{10}\text{B}$ concentration through autoradiography images in polycarbonate nuclear track detectors

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### ABSTRACT

The determination of the local concentration of boron in the different regions of tissue samples treated by Boron Neutron Capture Therapy (BNCT) could be made through the evaluation of the number of tracks forming autoradiography images. It is necessary to get a “standard” material containing a known amount of  $^{10}\text{B}$ , to correlate the number of tracks and boron concentration, i.e. to be used as a reference.

Different systems were tested in order to find a suitable standard. Films made of 2% agarose in boron solutions showed a homogeneous distribution of the  $^{10}\text{B}$  atoms in the material structure. This system is easy handled and its physical properties are satisfactory.

On the other hand, a small volume polycarbonate box was designed to contain  $^{10}\text{B}$  solutions of known concentration. This system showed a reduced number of background tracks and a promising behavior in many aspects. There is proportionality between track numbers per surface unit and  $^{10}\text{B}$  concentration, and between track numbers per surface unit and neutron fluence. Experimental results were compared to calculated values through formulas developed for thick samples autoradiography.

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### 1. Introduction

The capability of Solid State Nuclear Track Detectors (SSNTD) to register damage produced by heavy ions in a permanent way, has converted these materials into a powerful alternative for particle dosimetry. In the years after the first observations by Young (1958), nuclear tracks became the object of many and diverse research works and were also widely applied to a variety of fields.

When an ionizing heavy particle penetrates into a polymeric SSNTD, the ionization and excitation of the material atoms and the subsequent chemical reactions between the produced species may create a damaged zone of the material with broken polymeric chains. Narrow paths are thus formed along the ion's trajectory. These paths can be amplified by etching in an appropriate chemical attack solution, in order to visualize them with an optical microscope (Fleischer et al., 1975). Track pit shape is mainly determined by the ratio ( $V$ ) between the preferential attack rate  $V_T$  along the particle damaged trail and the bulk rate of attack ( $V_B$ , etching velocity in the non irradiated material).

If the ionizing particles are generated in “objects” containing heavy ions emitters and put in contact with the detector, an autoradiographic image of the object can be formed there by the produced tracks. This autoradiography image provides relevant information about the spatial distribution of heavy particle emitters in the specimen (Abe et al., 1986).

In particular, it can be used to determine the local distribution of  $^{10}\text{B}$  atoms in tissue samples coming from experimental animal models, or even from human patients, to be potentially treated by BNCT. In this case, the samples containing  $^{10}\text{B}$  in contact with the detectors must be irradiated with thermal neutrons to yield the capture reaction:  $^{10}\text{B}(n,\alpha)^7\text{Li}$ . So, the potential track generating particles are:  $\alpha$  particle with 1.47 MeV or 1.77 MeV energy (depending on the excitation level), and  $^7\text{Li}$  ion with 0.84 MeV and 1.02 MeV energy (Table 1). Protons produced in potential capture reactions with  $^{14}\text{N}$  atoms in tissue were also included in the table.

The comparison between histological and autoradiography images leads to a qualitative location of the boron atoms in the tissue sample (Altieri et al., 2008; Saint Martin et al., 2007). Moreover, the concentration of  $^{10}\text{B}$  in tissue samples may be inferred by measuring the track density in the detector.

Many techniques are applied at present to evaluate the amount of boron in tissue samples from BNCT investigations. Prompt gamma-ray spectroscopy, alpha spectrometry, inductively coupled

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**Table 1**

LET and Range values in polycarbonate were obtained with SRIM-8 (Ziegler et al., 1984) code for particles at initial conditions. V values in polycarbonate were calculated using expressions by Somogyi et al. (1976).

Particle	Energy (keV)	LET (MeVcm <sup>2</sup> /mg)	Range (μm)	V
Alpha <sub>0</sub>	1770	1.6588	8.24	1.400
Alpha <sub>1</sub>	1470	1.8069	6.769	1.509
Li <sub>0</sub>	1016	3.7424	3.815	4.968
Li <sub>1</sub>	840	3.5566	3.396	4.437
H+	580	0.3438	9.43	1.005

plasma mass/optical emission/atomic emission spectroscopy (ICP MS/OES/AES) are some examples (Wittig et al., 2008). These methods give the integral value of <sup>10</sup>B concentration in the sample. On the other hand, boron concentration can be locally evaluated when autoradiography techniques are applied, which is particularly convenient when the particle emitter is not uniformly distributed in the tissue section.

The following analytical expression, proposed in the literature, relates track density and boron concentration in the sample (Fleischer et al., 1975):

$$\rho = C(B) \frac{N_v \sigma_B \phi}{4} (R_\alpha \cos^2 \theta_\alpha + R_{Li} \cos^2 \theta_{Li}) \quad (1)$$

where  $C(B)$  is the concentration of boron atoms,  $N_v$  is the number of atoms per unit volume,  $\sigma_B$  is the neutron capture cross section, and  $\phi$  is the thermal neutron fluence.  $R_\alpha$ ,  $R_{Li}$  and  $\theta_\alpha$ ,  $\theta_{Li}$  are ranges and critical angles of the alpha particles and Li fragments respectively. Critical angles refer to the actual possibility of particles to be recorded in the detector and can be calculated in terms of V. Ions entering the detector with angles (measured between the ion's trajectory and the axis normal to the detector surface) larger than the critical angle, are not expected to be preferentially etched. Similar relationships are cited in Armijo and Rosenbaum (1967) and Durrani and Bull (1987).

An alternative approach to assess <sup>10</sup>B concentration in a given sample is to use some standard material with a known quantity of <sup>10</sup>B as a reference. The standard material (concentration  $C_s$ ) will produce a track density  $\rho_s$  in the area of the detector in contact with it, while a  $\rho_u$  track density will be measured in the area of the detector adjacent to the sample under investigation. Taking ratios of these quantities, the unknown concentration can be calculated as shown in Eq. (2).

$$\frac{\rho_u}{\rho_s} = \frac{C_u}{C_s} \quad (2)$$

As the reference system may be used to evaluate samples of materials with composition other than the standard one, it must be assumed that the range (in g cm<sup>-2</sup>) of the track producing particle in the standard and in the sample is virtually the same (Durrani and Bull, 1987). In fact the medium where the reaction occurs determines the energy loss of the produced particles in their trajectory to the detector surface and consequently the energy they arrive there with. This energy value, together with the incidence angle mentioned before, determine the possibility of observing a track in a certain detector with a given etching process.

Some desirable conditions for a standard in order to be used as a reference for neutron autoradiography are: homogeneous distribution of boron atoms in the material, minimal number of background tracks produced by the material, easy handling, and proportionality of track numbers with <sup>10</sup>B concentration and with thermal neutron fluence. Some authors have studied absorbing filter paper (Yanagie et al., 1999), liver homogenate (Fairchild et al.,

1986), boron doped Si (Bortolussi, 2007), etc. as standards, and used mainly cellulose nitrate and poly allyl diglycol carbonate (PADC) as track detectors, but not often employed polycarbonate as SSNTD. In this work, different systems were evaluated for their use as standards and calibration curves were obtained for polycarbonate detectors. Experimental results were also studied in relation with analytical expressions proposed in the literature.

## 2. Materials and methods

Lexan™ polycarbonate films of 250 μm thick were used as SSNTD. Different systems were evaluated for their use as standards:

- Preliminary experiments were performed using absorbing filter paper sheets (AFPS) soaked in borax solutions, as suggested in the previously mentioned literature. Filters of different shapes were used for this purpose: circles ( $\phi = 5$  mm) and rectangles (8 mm × 40 mm). They were stored at -20 °C for 24 h and then adhered on Lexan foils.
- Small boxes (Small Lexan Cases, SLCs) were assembled with Lexan foils. A simple scheme of the construction process is shown in Fig. 1. The two faces were obtained by cutting rectangular shapes of about 25 mm × 15 mm. Two L-shaped pieces were fixed to one face in order to give a thickness of 0.25 mm. The assembly was covered with the other face and all the elements were adhered with polycarbonate–chloroform solution. These boxes are able to contain a volume of about 100 μl. They were filled with <sup>10</sup>B solutions (99.99%) of concentrations ranging from 0 to 100 ppm.
- Homogeneous films were fabricated with LMP agarose (Low Melting Point: 65.5 °C, Promega™), in enriched boric acid solutions. Different 2% gels were prepared with <sup>10</sup>B concentrations from 0 ppm to 100 ppm. The agarose solutions in their liquid state were poured onto flat plastic molds and then covered with polycarbonate foils. After 24 h, the films were almost solid, so they were separated from the plastic mold. Weight corrections were applied in order to take into account changes in concentration due to evaporation.

Irradiations with thermal neutrons were carried out at the biomedical facility of the RA-3 reactor of the Ezeiza Atomic Center (CAE, Buenos Aires) with fluences of 10<sup>11</sup> n cm<sup>-2</sup>, 10<sup>12</sup> n cm<sup>-2</sup> and 10<sup>13</sup> n cm<sup>-2</sup>. As a matter of routine a calibration with a Self Powered Neutron Detector (SPND) is performed before each irradiation, to ensure that the samples are exposed to the same neutron fluence every time. During the irradiation, the flux is monitored in order to detect any variation that could occur. The fluence is measured with an uncertainty of 8%. More details about thermal neutron field characterization and dosimetry procedures are specified in Miller et al. (2009) and Pozzi et al. (2009).

After the irradiation process, the filters and agarose samples were removed from the polycarbonate detectors, which were etched in PEW alkaline solution (30 g KOH + 80 g ethyl alcohol + 90 g distilled water) at 70 °C for 2 min and then rinsed thoroughly with water. The SLCs were disassembled and the polycarbonate faces were then subjected to the same etching process.

The etching bulk velocity was previously determined by measuring the diameter of fission fragment tracks from a <sup>252</sup>Cf source. The obtained value was  $V_B = (19.2 \pm 0.6) \mu\text{m h}^{-1}$ . The etching time was chosen in order to obtain an appropriate average track diameter (easily observable), avoiding track overlapping and track density decrease due to layer removal. For that purpose both the track density and track diameter were measured for varying etching time in SLCs samples, corresponding to the 50 ppm and 10<sup>12</sup> n cm<sup>-2</sup> condition.

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