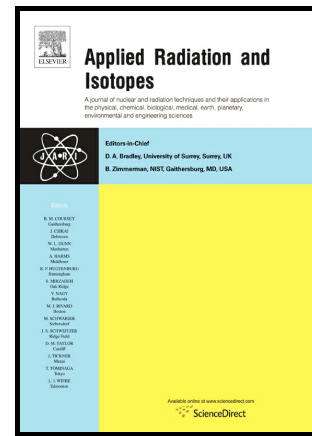


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Thorium-229 quantified in historical Thorium-228 capsules

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Abstract

Thorium-229 is a valuable, but scarce, radionuclide for nuclear clock applications or targeted alpha therapy. While it is mostly produced by the decay of ²³³U, ²²⁹Th can also be produced by neutron irradiation of ²²⁶Ra. At SCK•CEN, capsules containing mainly ²²⁸Th (by-product of ²²⁶Ra irradiation) were characterized to quantify the present amounts of ²²⁹Th, ²²⁸Th, ²²⁷Ac, ²²⁶Ra with high resolution gamma spectroscopy, after a decay period of 40 years in which ²²⁸Th has decayed. High purity ²²⁹Th was quantified, and after recovery using radiochemical separation processes, it can be used to support ongoing research.

1. Introduction

Thorium-229 is used as a generator of ²²⁵Ac and ²¹³Bi for Targeted Alpha-immunotherapy (TAT) (see e.g. Elgqvist *et al.*, 2014 and Jurcic *et al.*, 2015), and for nuclear clocks (see e.g. Karpeshin and Trzhaskovskaya, 2015 and Jeet *et al.*, 2015). Thorium-229 and its daughters do not occur in nature and belong to the decay series of ²³³U, bred from ²³²Th. The aged ²³³U stockpile is the only large source of ²²⁹Th in the world (about 9.3 GBq – 250 mCi), but is not likely to increase due to safety and security issues (Alvarez, 2013). Moreover, it is not radioisotopically pure due to ²³²Th and ²²⁸Th (decay of ²³²U) impurities. While for medical applications this is not a particular concern (besides additional shielding for ²²⁸Th), isotopic impurities in ²²⁹Th could be a significant issue for nuclear clocks (Stellmer *et al.*, 2015 and Von der Wense *et al.*, 2015).

The current driving need for ²²⁹Th is medical research and an increase in ²²⁵Ac/²¹³Bi production is needed (Nuclear Science Advisory Committee, 2015). Many new possible production routes for these three isotopes have been explored via particle accelerator or nuclear reactor (see e.g. Harvey, 2011; Hogle *et al.*, 2016; IAEA, 2009). While a few routes are promising, no relevant amounts have been produced yet, and the scarcity of ²²⁹Th and its daughter nuclides limits ongoing research. Therefore, attempts are made to retrieve ²²⁹Th from historical sources.

In 1969-1974, SCK•CEN and Union Minière conducted a project to produce a gram amount of ²²⁷Ac (the so-called *Actinium Programme*), by irradiating ²²⁶Ra in the high flux reactor BR2. Substantial amounts of ²²⁸Th were co-produced during irradiation, and SCK•CEN developed radiochemical methods to separate the complex mixture of target material and irradiation products (Monsecour *et al.*, 1974; Monsecour *et al.*, 1977; Baetsle *et al.*, 1972). During the *Actinium Programme*, ²²⁷Ac was separated after each irradiation cycle, while the ²²⁶Ra target material was purified and recycled for a

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