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A study on recovery of uranium in the anode basket residues delivered from the pyrochemical process of used nuclear fuel

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ACCEPTED MANUSCRIPT

| 1 | A study on recovery of uranium in the anode basket residues delivered |
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| 2 | from the pyrochemical process of used nuclear fuel |
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| 7 | |
| 8 | Abstract |
| 9 | In this study, the chlorination of uranium oxide (UO ₂) using ammonium chloride |
| 10 | and zirconium as chemical agents was conducted to recover the uranium in the anode |
| 11 | basket residues from the pyrochemical process of used nuclear fuel. The chlorination of |
| 12 | UO ₂ was predicted using thermodynamic equilibrium calculations. The experimental |
| 13 | conditions for the chlorination were determined using a chlorination test with cerium |
| 14 | oxide (CeO ₂). In the chlorination test, it was confirmed that UO ₂ was chlorinated into |
| 15 | UCl_3 at 320 °C, some UO_2 remained without changes in the chemical form, and ZrO_2 , |
| 16 | Zr ₂ O, and ZrCl ₂ were generated as byproducts. |
| 17 | |
| 18 | Key words: recovery of uranium, anode basket residue, pyrochemical process, |
| 19 | chlorination, UO ₂ |

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