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Computation of fundamental time-eigenvalue of the neutron transport equation

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ABSTRACT

A modified $\alpha - k$ power iteration method is presented for the prediction of time-eigenvalue(α) of the neutron transport equation. By developing a direct relationship between K-eigenvalue and α -eigenvalue, a new formula is introduced to estimate the value of α . Compared with the conventional method, it is not required to provide the initial values of α for the modified method. Since it is always difficult to guess the suitable initial values, the modified method is more convenient for solving time-eigenvalue problems. Computational experiences show that the accuracy of the modified method is the same as the conventional method.

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1. Introduction

Although several eigenvalue problems can be defined for the quasi-stationary neutron transport equation, the two most common formulations are effective fission multiplication factor (k_{eff}) and α - or time-eigenvalues, which are basic to the subject of nuclear reactor physics. The physical meaning and the properties of these eigenvalues were discussed in a very rich literature, e.g. (Bell and Glasstone, 1970; Larsen and Zweifel, 1974; Lewis and Miller, 1993).

In the K-eigenvalue problem, the fission source is artificially multiplied by a factor 1/k so as to obtain a balance between production and loss of neutrons and thus a steady state. The eigenvalues k_i are all real and positive. The largest value k_1 is called the effective fission multiplication factor $k_{\rm eff}$. The fluxes corresponding to the k_1 are positive everywhere, whereas the fluxes may be negative for other modes (Lewis and Miller, 1993, p. 46; Modak and Gupta, 2007). The power iteration (PI) method (Duderstadt and Hamilton, 1976) is commonly used to obtain the $k_{\rm eff}$.

The α -eigenvalue problem is defined in a different way. Let $\phi =$ $\phi(\vec{r}, E, \vec{\Omega}, t)$ denote neutron angular flux at point \vec{r} in the energy E and the direction $\vec{\Omega}$ at time *t*. In the α -eigenvalue problem, the flux is assumed to have an exponential time-dependence in the form

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$$\phi(\overrightarrow{r}, E, \overrightarrow{\Omega}, t) = \phi_{\alpha}(\overrightarrow{r}, E, \overrightarrow{\Omega}) \cdot e^{\alpha t}$$
(1)

Then, by insertion in time-dependent neutron transport equation, the eigenvalue equation can be obtained. The eigenvalue α appears in the form of a 1/v absorber. Like the K-eigenvalue problem, there are many possible eigenvalues α_i and corresponding eigenfunctions. Unfortunately, only a few general properties of the α -eigenvalue are known so far. The α_i need not be real and positive. They may be composed of a continuous spectrum and a discrete spectrum. In particular, it has been shown that under mild assumptions a dominant discrete eigenvalue α exists, which is real, larger than the real parts of all the other α , and whose associated eigenfunction is non-negative (Larsen and Zweifel, 1974; Zoia et al., 2014). Usually, the dominant discrete eigenvalue is called the fundamental eigenvalue.

The standard power iteration method cannot be used to compute the α -eigenvalue because the magnitude of fundamental α -eigenvalue is not the largest (Modak and Gupta, 2007). To find the fundamental α -eigenvalue, a $\alpha - k$ power iteration method has been developed in some recent papers, and adopted by many neutron transport codes (Briesmeiser, 2000; Ye et al., 2007; Zoia et al., 2014; Zoia et al., 2015). Although the power iteration method dominates the field of eigenvalue computation, other methods have emerged, for instance, implicitly restarted Arnoldi method (IRAM) (Lathouwers, 2003; Kópházi and Lathouwers, 2012) and nonlinear solution method (Fichtl and Warsa, 2013). Kópházi and Lathouwers (2012) refers to the fact that the IRAM is





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applicable only to subcritical systems. Fichtl and Warsa recast the α -eigenvalue problem as a nonlinear problem, and solve this nonlinear problem by nonlinear solvers.

In order to find the α -eigenvalue by the $\alpha - k$ power iteration, it is required to provide a initial value of α (see Section 2.2 for details). It is always difficult to guess the suitable initial value. Therefore, an attempt has been made here to design a new $\alpha - k$ power iteration method for obtaining the α -eigenvalue of the neutron transport equation. The initial value is no longer needed for the new method.

The remainder of this paper is organized in the following manner. Section 2 briefly describes the K- and α -eigenvalue equations, and the conventional methods. Section 3 details the new computation scheme developed by this work. Section 4 presents some numerical results for typical problems to demonstrate the validity and efficiency of the new method. Finally, Section 5 gives conclusions.

Since only "prompt" α -eigenvalue problem is considered in this paper, the delayed neutrons are neglected. So, in the following equations, the notation v_p is adopted to denote the average number of prompt neutrons per fission.

2. Eigenvalue problems and conventional methods

2.1. Eigenvalue problems

The α -eigenvalue equation can be derived from the timedependent neutron transport equation given below:

$$\phi_{X}(\overrightarrow{r}, E, \overrightarrow{\Omega}) = \phi_{X}(\overrightarrow{r}, E, \overrightarrow{\Omega}'), \quad \overrightarrow{r} \in \partial V_{R}, \quad \overrightarrow{\Omega} \cdot \overrightarrow{n} < 0$$
(6)

for the bare and reflected boundaries, respectively. Here $\vec{\Omega}'$ is the reflected direction from $\vec{\Omega}$, and the subscript *x* is equal to α or *k*.

2.2. Conventional methods

Since the K-eigenvalues are real positive and the fundamental Keigenvalue is the greatest, it is invariably solved by the method of power iteration. This method is well known and can be found in a very rich literature (Bell and Glasstone, 1970; Lewis and Miller, 1993; Du and Zhang, 1988), so further details are not presented.

Although the standard power iteration method cannot be used to find the α -eigenvalue (Modak and Gupta, 2007), a $\alpha - k$ power iteration method has been developed. By introducing a fictitious parameter k dividing the fission term, α -eigenvalue Eq. (3) can be written as follows:

$$\vec{\mathcal{Q}} \cdot \nabla \phi_{\alpha} + \left(\Sigma_{t} + \frac{\alpha}{\nu}\right) \phi_{\alpha} = \int_{0}^{\infty} dE' \int_{4\pi} d\mathcal{Q}' \Sigma_{s} \left(\vec{r}, E' \to E, \vec{\mathcal{Q}}' \cdot \vec{\mathcal{Q}}\right) \\ \times \phi_{\alpha} \left(\vec{r}, E', \vec{\mathcal{Q}}'\right) + \frac{1}{k} \frac{\chi(E)}{4\pi} \int_{0}^{\infty} dE' \nu_{p} \Sigma_{f}(\vec{r}, E') \\ \times \int_{4\pi} d\mathcal{Q}' \phi_{\alpha} \left(\vec{r}, E', \vec{\mathcal{Q}}'\right)$$
(7)

$$\frac{1}{\nu}\frac{\partial\phi}{\partial t} + \vec{\Omega}\cdot\nabla\phi + \Sigma_t\phi = \int_0^\infty dE' \int_{4\pi} d\Omega' \Sigma_s\left(\vec{r}, E' \to E, \vec{\Omega}' \cdot \vec{\Omega}\right) \phi\left(\vec{r}, E', \vec{\Omega}', t\right) + \frac{\chi(E)}{4\pi} \int_0^\infty dE' \nu_p \Sigma_f(\vec{r}, E') \int_{4\pi} d\Omega' \phi\left(\vec{r}, E', \vec{\Omega}', t\right)$$
(2)

here the well known notations are used. If Eq. (1) is substituted in Eq. (2), the time derivative term is replaced by $\alpha/v\phi$ and then the common exponential term $e^{\alpha t}$ is removed, leading to the time-independent α -eigenvalue equation:

which becomes a standard *k*-eigenvalue equation, the parameter α being though unknown. The basic strategy is to seek then the value α for which k = 1 (Zoia et al., 2014).

So, the α -eigenvalue can be found by the iterative scheme as

$$\vec{\Omega} \cdot \nabla \phi_{\alpha} + \left(\Sigma_{t} + \frac{\alpha}{\nu}\right) \phi_{\alpha} = \int_{0}^{\infty} dE' \int_{4\pi} d\Omega' \Sigma_{s} \left(\vec{r}, E' \to E, \vec{\Omega}' \cdot \vec{\Omega}\right) \phi_{\alpha} \left(\vec{r}, E', \vec{\Omega}'\right) + \frac{\chi(E)}{4\pi} \int_{0}^{\infty} dE' \nu_{p} \Sigma_{f}(\vec{r}, E') \int_{4\pi} d\Omega' \phi_{\alpha} \left(\vec{r}, E', \vec{\Omega}'\right)$$
(3)

The K-eigenvalue problem is formulated by assuming that v_p , the average number of neutrons per fission, can be adjusted to obtain a time-independent solution to Eq. (2). Hence the K-eigenvalue equation may be written as follows:

follows (Du and Zhang, 1988):

1. α_1 and α_2 which are two initial values of α , are guessed. Then k_1 and k_2 are found by the power iteration from Eq. (7),

$$\vec{\Omega} \cdot \nabla \phi_k + \Sigma_t \phi_k = \int_0^\infty dE' \int_{4\pi} d\Omega' \Sigma_s \left(\vec{r}, E' \to E, \vec{\Omega}' \cdot \vec{\Omega}\right) \phi_k \left(\vec{r}, E', \vec{\Omega}'\right) + \frac{1}{k} \frac{\chi(E)}{4\pi} \int_0^\infty dE' \nu_p \Sigma_f(\vec{r}, E') \int_{4\pi} d\Omega' \phi_k \left(\vec{r}, E', \vec{\Omega}'\right)$$
(4)

Finally, the common boundary conditions for the two problems are formed as

$$\phi_{X}(\overrightarrow{r}, E, \overrightarrow{\Omega}) = \mathbf{0}, \quad \overrightarrow{r} \in \partial V_{B}, \quad \overrightarrow{\Omega} \cdot \overrightarrow{n} < \mathbf{0}$$
(5)

respectively.

2. Based on (k_{n-2}, α_{n-2}) and (k_{n-1}, α_{n-1}) , the new estimate of α , α_n , is obtained by a linear extrapolation (8). Subsequently, k_n can be found from Eq. (7).

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